



# D-<sup>3</sup>He Fueled Fusion Devices as Step Towards Total Fusion Safety

**L. El-Guebaly**  
Fusion Technology Institute  
UW - Madison

and

**M. Zucchetti**  
Politecnico di Torino  
Torino, Italy

ICENES – 2007

13<sup>th</sup> International Conference on Emerging Nuclear Energy Systems

June 3–8, 2007  
Istanbul, Turkey



## History & Background Info

- In late 1980s, fusion **safety** stimulated worldwide research for **fuel cycles other than D-T**.
- Advanced cycles (such as D-D, **D-<sup>3</sup>He**, p-<sup>11</sup>B, and <sup>3</sup>He-<sup>3</sup>He) **do not require breeding large amount of T** (~56 kg/y for 1 GW of D-T fusion power).
- Some advanced cycles (e.g., D-<sup>3</sup>He) are **not completely aneutronic** due to side D-D reactions.
- However, **neutron wall loading can be kept low** (by orders of magnitude) compared to D-T fueled plants with same output power.
- **Attractive features for D-<sup>3</sup>He fuel cycle include:**
  - No T breeding blanket.
  - All components are permanent, meaning no need to replace FW/shield during entire plant lifetime (~ 50 y).
  - Potential for direct energy conversion.
  - **Low activity, decay heat, and radwaste levels.**
  - Low releasable radioactive inventory from credible accidents.
- **Concerns for D-<sup>3</sup>He fuel cycle :**
  - <sup>3</sup>He availability.
  - Higher plasma ion temp (50-100 keV) compared to D-T (10-20 keV).

# Objectives

---

- Apply most recent radwaste management schemes to D-<sup>3</sup>He fueled devices: ARIES-III power plant and Candor experiment\*.
- Compare radiological aspect of D-<sup>3</sup>He and D-T fuel cycles and highlight differences.

\* L. El-Guebaly and M. Zucchetti, "Recent Developments in Environmental Aspects of D-<sup>3</sup>He Fuelled Fusion Devices," to be published in Fusion Engineering and Design.

# Handling Fusion Radwaste is Important to Future of Fusion Energy

---

- Majority of fusion power plants designed to date focused on **disposal** of active materials in repositories, adopting fission waste management approach preferred in 1970's.
- **New Strategy**: Develop new framework for fusion: nothing should be disposed of in ground, instead **recycle and/or clear** all active materials, if technically and economically feasible.
- **Why?**
  - Limited capacity of existing low-level waste repositories
  - Political difficulty of building new repositories
  - Tighter environmental controls
  - No radwaste burden for future generations.
- **Impact**: Promote fusion as **waste-free** source of energy.



# Fusion Designs Should Adopt **MRCB** Philosophy

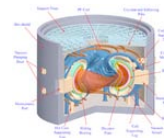
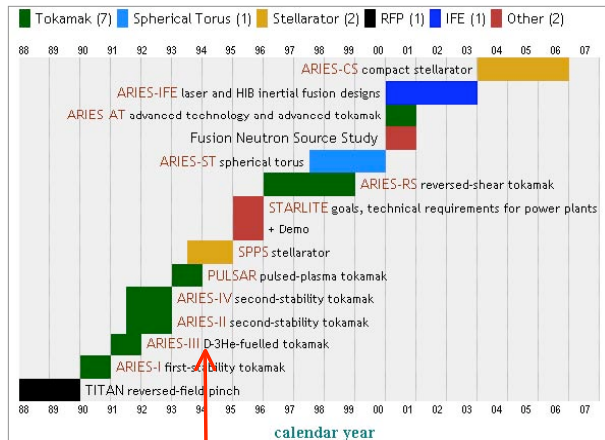
- M** – Minimize volume of active materials by design or by employing advanced fuel cycle.
- R** – Recycle, if economically and technologically feasible.
- C** – Clear slightly-irradiated materials.
- B** – Burn active byproducts, if any, in fusion devices\*.

\* L. El-Guebaly, "Managing Fusion High Level Waste – a Strategy for Burning the Long-Lived Products in Fusion Devices," *Fusion Engineering and Design*, **81** (2006) 1321-1326.

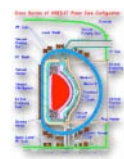


# ARIES-III Power Plant Selected for Radwaste Assessment

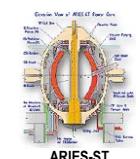
### ARIES Project Timeline



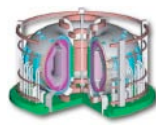
ARIES-CS



ARIES-AT



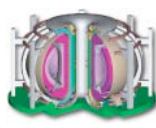
ARIES-ST



ARIES-I



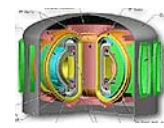
ARIES-II



ARIES-IV

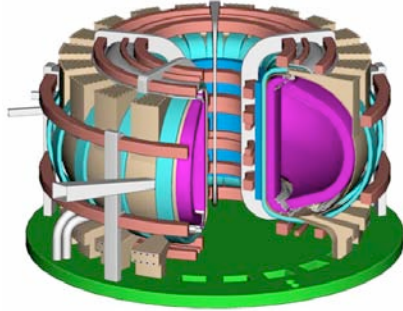


SPPS



ARIES-RS

# ARIES-III Power Plant

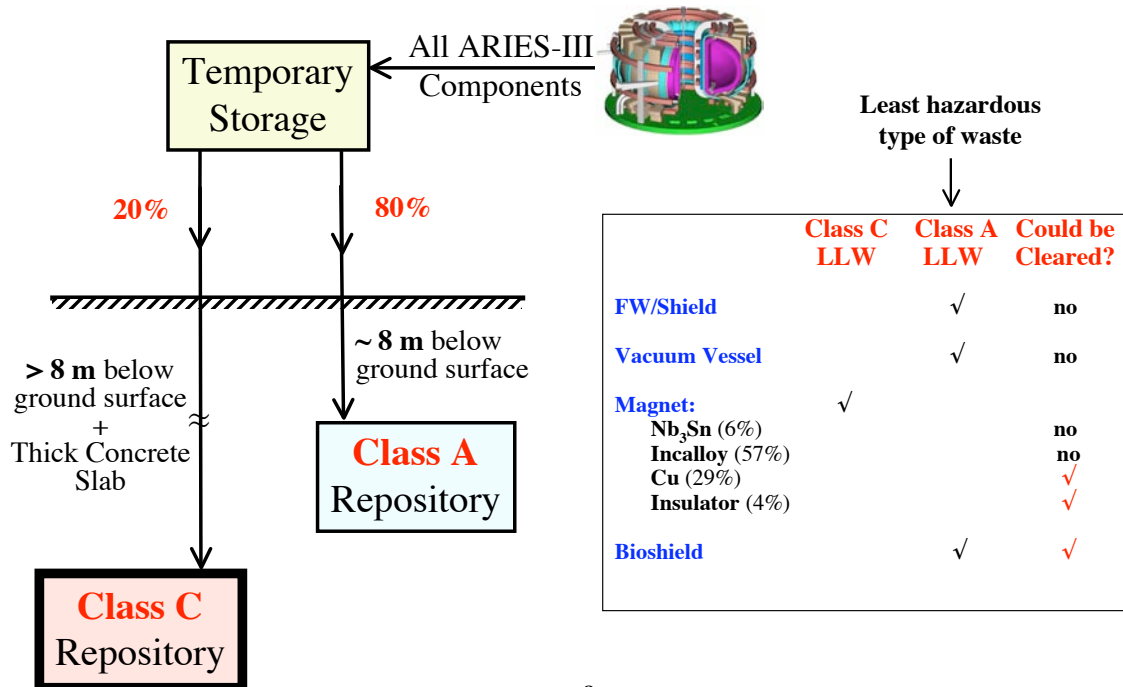


1000 MW<sub>e</sub> Output Power  
 7.5 m Major Radius  
 D-<sup>3</sup>He Neutron Source:  
     30% 14.1 MeV n's  
     70% 2.45 MeV n's  
 0.1 MW/m<sup>2</sup> Average NWL  
 Ferritic Steel Structure  
 Organic Coolant with 44% η<sub>th</sub>  
 40 FPY Permanent Components  
 85% Availability

7



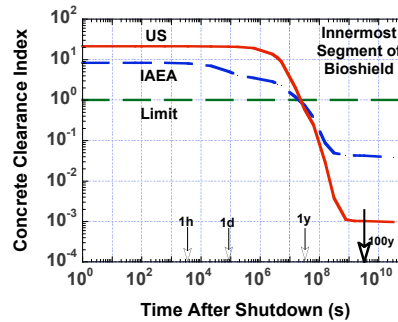
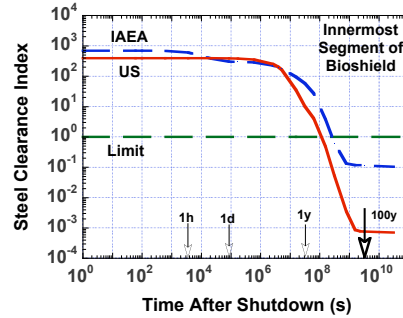
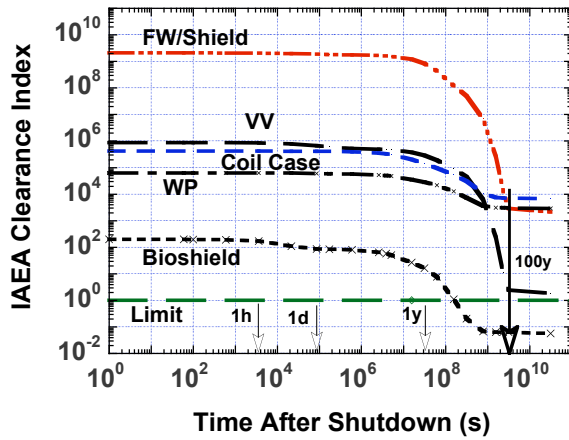
## ARIES-III LLW Classification for Geological Disposal



8



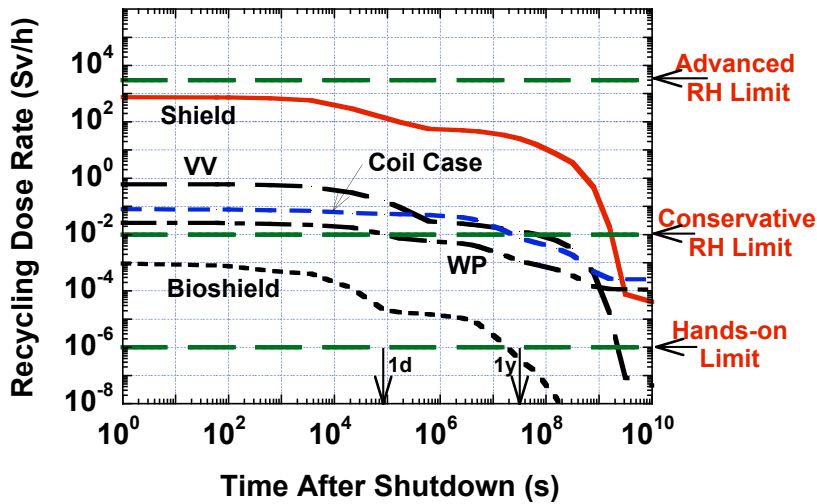
# 85% of ARIES-III Radwaste can be Cleared in < 10 y after Decommissioning



9

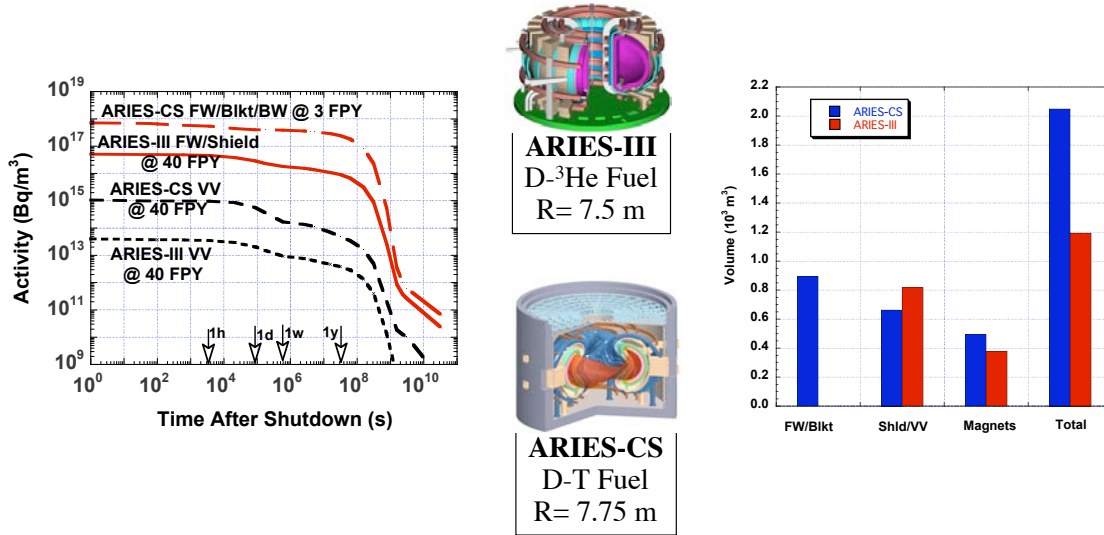


# All ARIES-III Components can be Recycled in < 1 y Using Advanced and Conventional RH Equipment



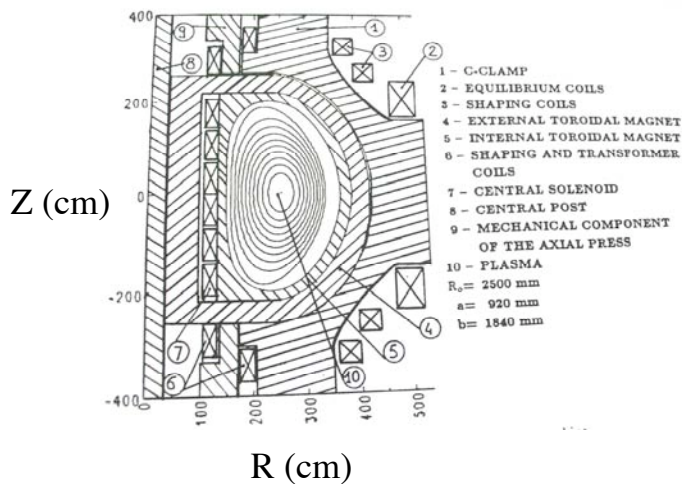
10

# Comparison of D-<sup>3</sup>He and D-T Fueled Power Plants of Comparable Major Radii



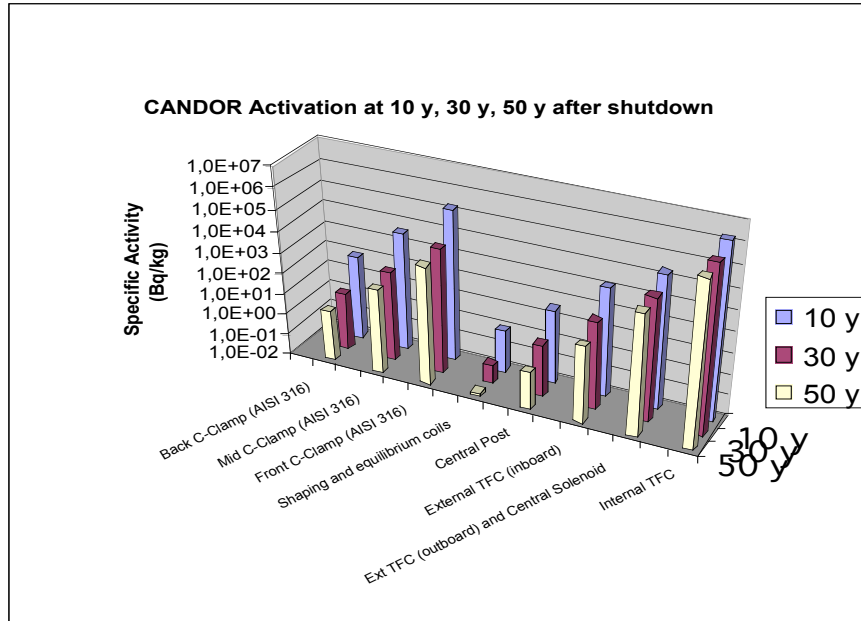
ARIES-III radwaste inventory is ~50% of ARIES-CS'

## Candor Experiment



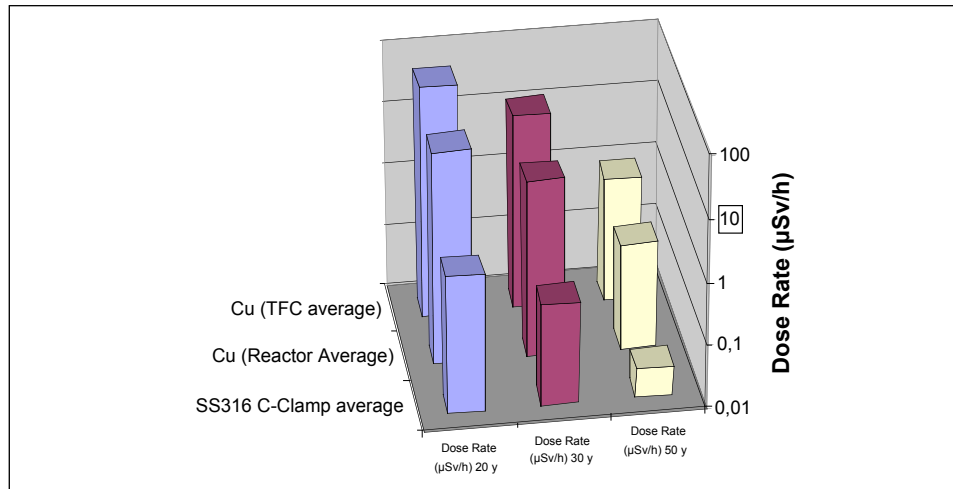
2.5 m Major Radius  
 Cu Magnet  
 D-T Trigger to Reach Ignition  
 D-<sup>3</sup>He Neutron Source:  
 26% 14.1 MeV n's  
 74% 2.45 MeV n's  
 1.2 MW/m<sup>2</sup> Average NWL

# Candor Specific Activity



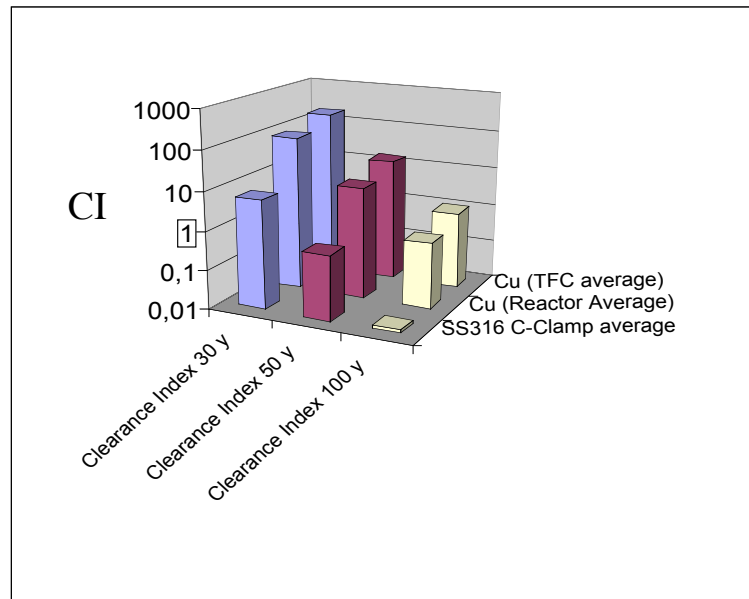
13

# Hands-on Recycling of Candor Components is Feasible within 10-30 y



14

## All Candor Components can be Cleared in 50-100 y



15

## Conclusions

- **Recycling and clearance** of all components should be essential goal of fusion studies to minimize radwaste stream.
- Advanced  $D-^3He$  fuelled designs offer further **step toward intrinsic safety and environmental goals**.
- For  $D-^3He$  fuelled **ARIES-III power plant**:
  - All in-vessel components qualify as Class A waste, the least hazardous type based on U.S. guidelines.
  - All components can be recycled using conventional and advanced remote handling equipment.
  - Bioshield contains traces of radioactivity and can be cleared from regulatory control after relatively short period of time ( $\sim 10$  y).
- $D-^3He$  fuelled **Candor experiment reaches zero-waste option** as all wastes can be cleared within 100 y.
- Low neutron production of  $D-^3He$  fuel helps **overcome some of engineering and material hurdles to fusion development**.
- Advanced fuel cycle **development should be carried out** in parallel with current mainstream fusion pathway that primarily focuses on D-T tokamaks.

16